

CHAPTER 17.0 - QUALITY ASSURANCE

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The quality assurance program for the Byron and Braidwood Stations is conducted in accordance with the Quality Assurance Program for Nuclear Generating Stations. This program was initially submitted to the Nuclear Regulatory Commission in June 1975, as Topical Report CE-1. By letter of December 29, 1975, the NRC's Chief of Quality Assurance Branch, Division of Reactor Licensing, informed Commonwealth Edison's Executive Vice President, Mr. W. B. Behnke, that this Topical Report CE-1 was an acceptable program for the design, procurement, construction and operations activities within Commonwealth Edison's scope of work for nuclear power plants. Instructions were given to reference this Topical Report in Section 17 of License applications.

Currently Topical Report NO-AA-10 is the basis for the Quality Assurance Program at the Byron/Braidwood Stations. |

Chapter 2 and Appendix C of the above-referenced topical report provides for compliance with certain regulatory guides and ANSI standards. Any exceptions taken to the requirements of these guides are found in Appendix A of this UFSAR. Otherwise, Byron/Braidwood Stations are in compliance with the guides and ANSI standards. |

Subsection 3.2.1.1 of the UFSAR states that Safety Category I systems or portions of systems and components meet the requirements of Appendix B to 10 CFR 50. Those structures, systems, and components classified as Safety Category I, therefore, are under control of the QA program and are listed in Table 3.2-1. Where only portions of a system are classified as Safety Category I, the P&IDs identify the boundary between Safety Category I and Safety Category II.